The Smallest SMRs

Ken Kozier & David Sears AECL-Chalk River Laboratories Western Focus Seminar/33rd Annual CNS Conference Saskatoon, SK, 2012 June 10-13





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Outline

- What is a <u>very Small Modular Reactor</u> (SMR)?
- Why they may be of interest to Canada; what is the market?
- What's been done <u>before</u> elsewhere & in Canada?
 - -Slowpoke Energy System
 - -Nuclear Battery
- What are others doing <u>now</u>?
 - Emphasis placed on <u>transportable</u> reactor core modules & <u>autonomous/unattended</u> operation
- What is the <u>future</u> potential for very small reactors in Canada?



What do we mean by a '<u>very</u> SMR' (or 'Micro Reactor')?

- <u>Small</u> Reactor: defined by power output
- IAEA: <300 MWe/~1000 MWt [SMR = Small & Medium Reactors]</p>

CNSC: <200 MWt/~70 MWe [RD-367 Design of Small Reactor Facilities]

- A risk-informed "<u>flexible/graded</u>" approach to licensing, but the same process irrespective of size
- For this talk: very SMR is <<10-30 MWe/<<30-100 MWt (still a <u>BIG</u> subject)
- May be small in physical size at low power output
 - Examples: 200 kWe/1.1 MWt EBR-I NaK fast reactor 1951-63; 52 kg HEU
 - -0.65 kWe/45 kWt SNAP-10A space reactor 1965





Canadian remote SMR market

- Electricity & heat in remote off-grid locations currently served by diesel generators
 - >300 isolated communities that rely on diesel generators for electricity; 50% in the North + <u>mines</u> + military installations
 - Energy use/capita & cost increases with latitude
 - <u>Diesel fuel cost</u>: \$5 /litre in CFS Alert 82° N [2007 Jan.]
 - Most Canadian Arctic community needs are <<10 MWe, somewhat more (~2 to 38 MWe) for mines
- Sustainable economic & social development of Canada's North requires stable, secure, low-cost & environmentally responsible energy

<u>Value proposition</u>: Reliable low-cost energy \rightarrow <u>mining development</u> \rightarrow jobs \rightarrow social advancement \rightarrow stable communities \rightarrow enhanced sovereignty

 Current reliance on fossil fuels is <u>vulnerable</u>: fuel costs, supply logistics, security of supply of 'Arctic' fuels, environmental concerns

May 29, 2011

Global warming jeopardizing ice highways, study says By Nathan Vender Klippe From Monday's Globe and Mail

Canada will lose winter-road access to nearly 400,000 square kilometres of land by mid-century, UCLA researchers predict



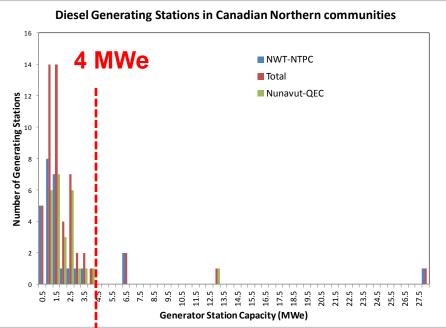
Arctic Canada Diesel Fuel Users Active Mineral Exploration Project Northwest Territories and Nunavu Resource Extraction National Defence: CFS Alert 82° N •

- Remote Communities
 - *Example*: Grise Fiord: 76° N, pop.141
 - Average annual T=-16.5° C
- **UNRESTRICTED / ILLIMITÉ**

Diesel electricity generating stations in Canadian Northern <u>communities</u> [Wikipedia]

- 53 communities in NWT & Nunavut: total ~100 MWe
- 49 generating stations <4 MWe [~160 0.5-MWe units]
- Need small units for redundancy, transportability, baseload operation (no information on daily & seasonal load factors)
- <u>Energy costs</u>: Qulliq Energy Corporation, <u>Nunavut</u>
 - Electricity: Lowest (Iqaluit) **\$0.5239 /kWh** Highest (Kugaaruk)
 <u>\$1.0271 /kWh</u> (National Energy Board; after Govt. subsidy)
 - Fuel <u>subsidies</u> consume ~<u>\$200 million</u> per year; ~20% of the entire <u>Govt. of Nunavut</u> budget. (pop. 31,906 [2011]; <u>\$6270 per person</u>)
 - Total energy cost: <u>~\$11,000/5,440 litres</u> per person; 26% for electricity
- Diesel fuel supplied by ship from Montreal
- Only alternative is by <u>air</u> if early freeze-up
- Only 3 suitable airstrips
- A single air shipment cannot satisfy <u>daily</u> fuel requirements in most communities
- <u>92%</u> of Nunavut \$1.4B budget comes from Federal Govt.

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Nunavut

BCEODICON.

The past: Early US Army Corps of Engineers 'Packaged' Reactor Program: 1954-77

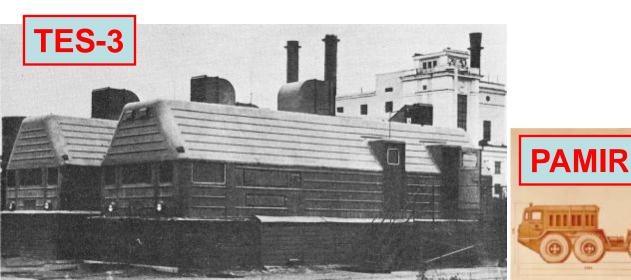
- 8 reactors built mainly by American Locomotive Co.: mostly High Enriched Uranium (HEU) PWRs
- SM-1: 2-MWe Ft. Belvoir, VA, <u>1957-4-4</u> to 1973
- SL-1: 300-kWe BWR NRTS, Idaho, 1958-61
- <u>PM-2A</u>: 2-MWe Camp Century, Greenland, 1960-64
- ML-1: 253-kWe gas turbine, 15 tons, NRTS, 1962-63
- **<u>PM-1</u>**: **1.25-MWe**, Sundance, WY, 1962-68
- PM-3A: 1.75-MWe, McMurdo Sound, Antarctica, 1962-72
- SM-1A: 2-MWe, Ft. Greely, Alaska, 1962-72
- MH-1A: 10-MWe + H₂O, on Sturgis, Panama Canal, 1968-77





Some Russian small reactors

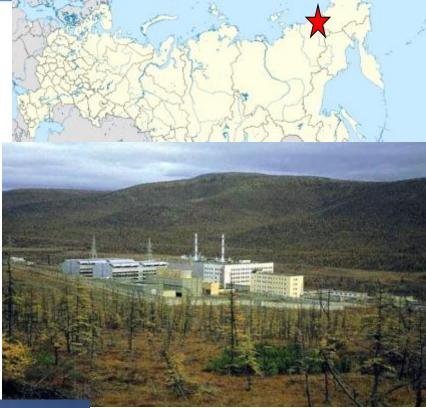
- AM-1: 5 MWe/30 MWt IPPE, Obninsk <u>1954</u>-2002: RBMK precursor
- GAMMA: 6 kWe/220 kWt Bi-Te-Se TE's; Ti vessel
 [6 km ocean depth],1981-92, Kurchatov Institute, Moscow
- Russian space reactors: 37 launched 1967-88
 - -ROMASHKA [0.8 kWe/40 kWt], BOUK: <0.5 kWe TE
 - TOPAZ: <10 kWe thermionic; COSMOS-1867 in 1987 ~11 months unattended operation record
- Portable: TES-3 2 MWe 1961-65; PAMIR 0.6-MWe 1985-86





Russia's existing Bilibino Nuclear Power Plants

- 4x12 MWe EGP-6 reactors
- Most northern Nuclear Power Plants 68° N
- H₂O-cooled graphite-moderated channel reactors based on AM-1
- 1973 ...; electricity & heat for <u>gold</u>
 <u>mine</u>, greenhouse & town of 4,500
 - Provides ~70% of energy needs
 - Average load 15-25MWe
 - -670 plant staff!!!; 40% of electricity cost





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Early Canadian 'CANDU-ish' Very Small Power Reactor experience

NPD

Whiteshell

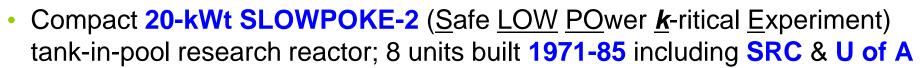
Laboratories

- Nuclear Power Demonstration (NPD) Rolphton, Ontario
 - -22 MWe [19.5 MWe net]; 1st PHWR
 - CGE/AECL/OH partnership
 - 1st critical <u>1962</u> Apr. 11; S/D 1987
- Whiteshell Reactor (WR-1), Pinawa, Manitoba Western Focus
 - 60 MWt [13-15 MWt to heat Whiteshell Laboratories (WL)]
 - prototype Organic Cooled Reactor built by Canadian General Electric
 - -1st critical **1965** Nov. 1; S/D **1985** May

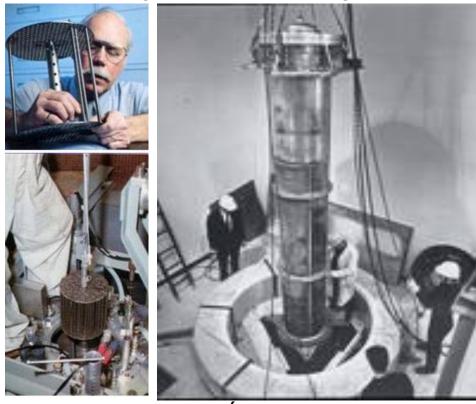
WR-1



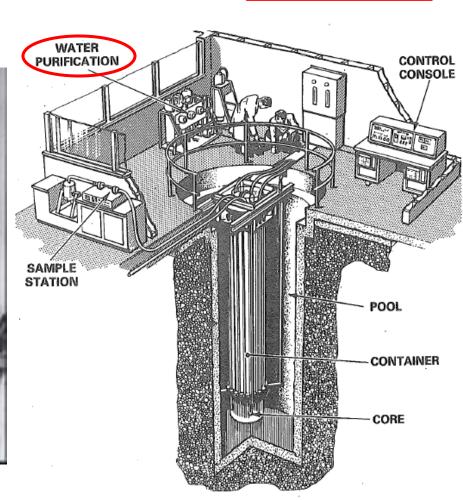
Scaling up from existing small reactors



- Licensed for unattended operation for up to 24 h
- Inherent/passive safety features
- -0.88 kg ²³⁵U HEU; 1.12 kg ²³⁵U LEU



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Western Focus



2-MWt SLOWPOKE Demonstration Reactor (SDR) at WL Western Focus

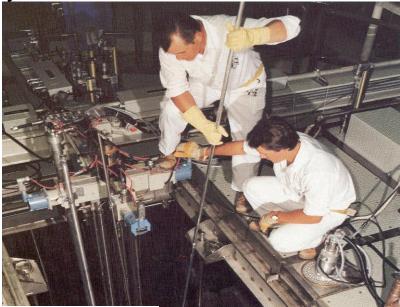
- Interest in heating reactors ~1984 [CFS Alert 2-MWt]; Local Energy Systems business unit formed under W.T. Hancox
- Construction started 1985 spring; 1st criticality 1987 July 15
- 4.3-m diameter x 10-m deep steel-lined pool; 100 kg U
- Operated briefly for physics & thermalhydraulics tests





Local Energy Systems

A BUSINESS UNIT OF ATOMIC ENERGY OF CANADA LIMITED





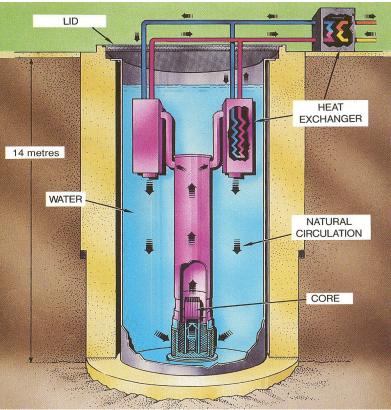


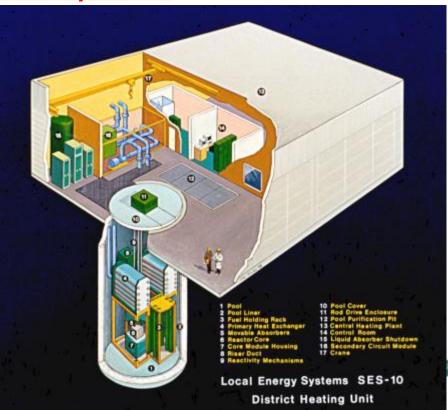
10-MWt SLOWPOKE Energy System (SES-10)

Commercial follow-up to SDR for <u>urban</u> low-temperature hot-water heating systems (85° C); up to 150,000 m² of floor area, 1500 apartments

- Canadian interest: Université de Sherbrooke initially & University of Saskatchewan later

- ~1100 kg of fuel every 4 to 6 years (removed after 10-12 years) Western Focus
- 6-m diameter x 13.5-m deep steel-lined pool
- Unpressurized; natural circulation; <u>unattended operation</u> for weeks



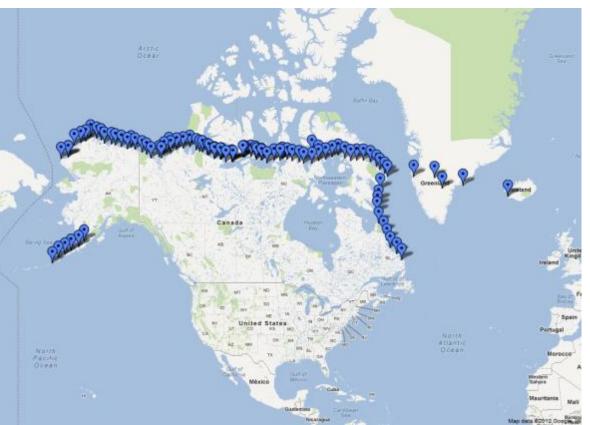


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North Warning System (NWS) application

(US/Canada agreement signed at 1985 'Shamrock Summit')

- Replacement for aging NORAD DEW line to detect cruise missiles
- 13 minimally attended Long Range Radar & <u>39 unattended</u> Short Range Radar (15 kWe) sites across 7000 km at ~70° N latitude
- Extraordinary <u>reliability</u> requirement >99.99% (<1 hr/a) for 20 years
- Helicopter transport (<12 Mg); hazardous 'whiteout' conditions

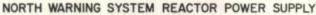




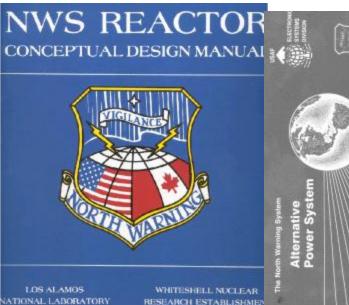
Diesel vs. F	leactor	
	Diesel	Reactor
Number of Units	4	1
Power/Unit	15/40 kWe*	15/40 kWe
System Weight	130 Tons	22 Tons
Maintenance Interval (Mos)	3	12
Overhaul Interval (Years)	1	20
Annual Fuel Consumption	18,000Gals	s. 0
Refuel Trips/Year	26	0 ·
Life Cycle Cost	\$3,109K	\$2,238K

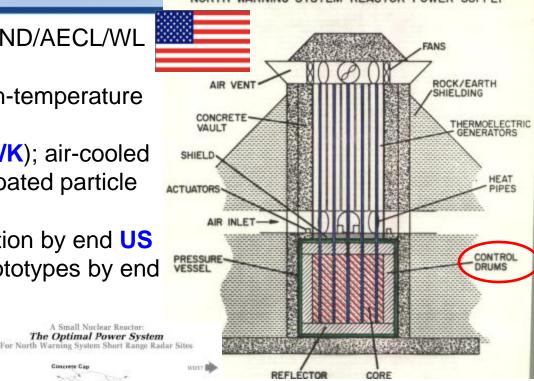
Three units required for maintenance period.

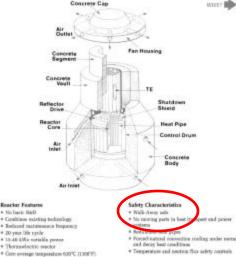
NWS reactor concept 15 kWe net (200 kWt)



- 1984 joint USAF/DOE/LANL & DND/AECL/WL investigation
- Innovative hybrid of space & high-temperature reactor concepts
- Heat pipe cooled reactor (19 SS/K); air-cooled **TE** conversion system, **TRISO** coated particle fuel [no containment building]
- Ambitious schedule: demonstration by end US FY86 [~\$24 million]; deploy 2 prototypes by end FY87; remaining SRRs FY88-91







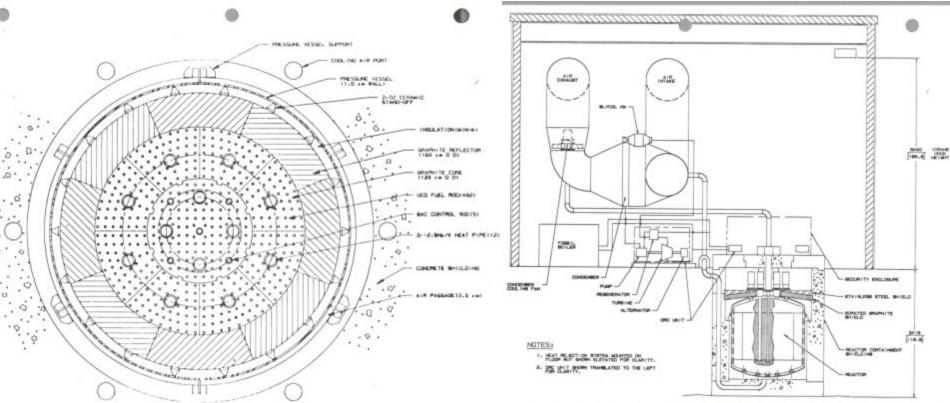
A Small Nuclear Reactor:



Compact Nuclear Power Source (CNPS)



- NWS power requirement increased to 20 kWe [40 kWe maintenance]
- More efficient power conversion system toluene Organic Rankinecycle Engine (ORE) [135 kWt; 64 kg U; 12 Zr-2.5%Nb/K heat pipes]
- Much shorter heat pipe condenser region due to ORE vaporizers
- 5 B₄C control rods instead of 18 rotating reflector drums



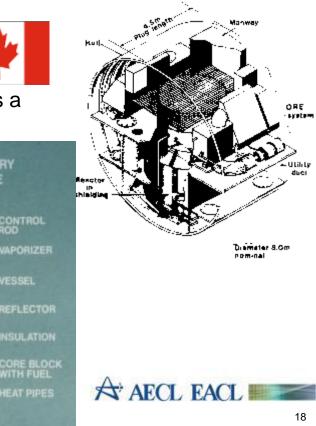
"Nuclear Battery" (W.T. Hancox) Concept Evolution

- Joint CNPS project ceased ~1986-87
- LANL: 1987-88 CNPS critical expts.
- WL: 500-kWe <u>Air Independent Propulsion</u> (AIP) System for Canadian Submarine Acquisition Project (CASAP) version for Canadian Navy
 - ORE-based design much more compact (seawater condensers)

<u>Remote Village Electricity</u>: 600 kWe/2.4 MWt

- Solid-state graphite core block [2.5-m x 2-m] functions as a thermal energy storage cell from which useful energy is passively extracted using 159 Nb-1%Zr/K heat pipes
- Complementary follow-up to SES-10, but at earlier stage of design
- No refuelling for core lifetime: <u>15 years</u>
- LEU TRISO fuel (0.5-mm UO₂ kernels);
 burnable poisons; ~550° C core
- High-Grade Steam Heat Source
- R&D work ceased in 1989
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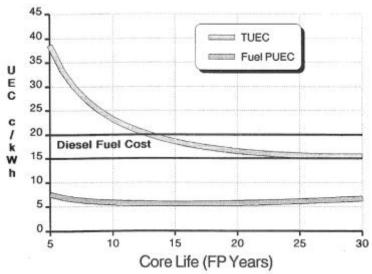


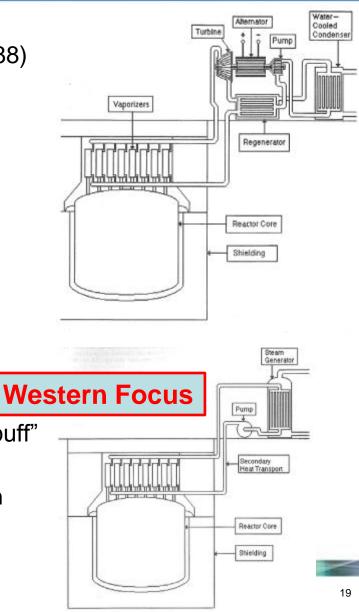


Canadian Nuclear Battery market potential

<u>Village electricity</u>:

- Competitive with diesel fuel at ~\$0.2 /kW_eh (1988)
- -~150 units





- <u>Alberta Oil Sands</u>: ~22% of global recoverable
 petroleum (1988) [2.3x10¹⁰ m³ & 6.8 GJ/m³ "huff-&-puff" process]
 - High-temperature steam conditions similar to an OCR [secondary coolant loop ≥370° C]
 - -<u>**140,000</u>** units!!! [**1 NB** = 13,149 MWd]</u>

Nuclear Battery component technology R&D at WL

Gravity-Assist Zr-Potassium Heat Pipe

- Demonstrated heat transfer performance up to 1.1 kWt/cm² at 500°
- Near-full-scale heat pipe transported ~19 kWt





Sundstrand REMCOM ORC

Combined Rotating Unit

Toluene Organic Rankine-cycle Engine (ORE)

 Demonstration 1-kWe toluene ORE unit operated at WL for 2400 h, including 650 h single continuous run with simulated remote startup

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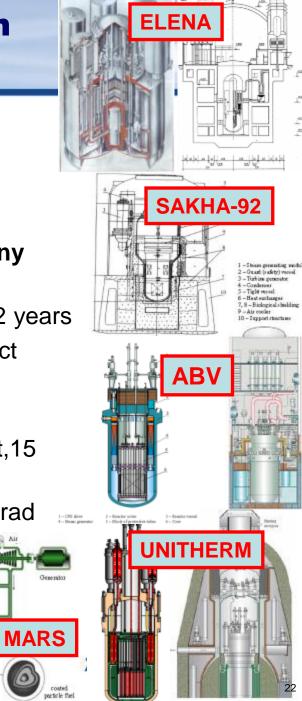
\$5,000 rpr



The <u>smallest</u> current Russian SMR designs & concepts

- <u>ELENA</u>: 68 kWe[TE]/3.3 MWt I(*Integral*)PWR for district heating, Kurchatov Institute (KI), Moscow
 - Transportable, autonomous/unattended
 - -21.7 year core lifetime; scale-up from GAMMA
- <u>SAKHA-92</u>: <u>1 MWe</u>/7 MWt IPWR, 25 years, OKBM, Nizhny Novgorod, transportable by rail, sea or truck
- <u>ABV-3</u>: 2.5 MWe/16 MWt IPWR, OKBM Afrikantov, 10-12 years
- <u>UNITHERM</u>: 6 MWe/30 MWt (or 2.5 MWe + 20 MWt district heating), IPWR, 20-year lifetime, NIKIET/RDIPE, Moscow
 - Transportable, autonomous/unattended
- MARS: 6 MWe/16 MWt, KI, <u>Molten Salt Reactor</u> concept,15 or <u>60 years</u>, <u>autonomous</u>, air turbine
- <u>VKT-12</u>: 12 MWe <u>BWR</u> 10 years, RDIPE/RIAR, Dimitrovgrad

Salt-az



Some more very SMR designs & concepts



CAREM-25

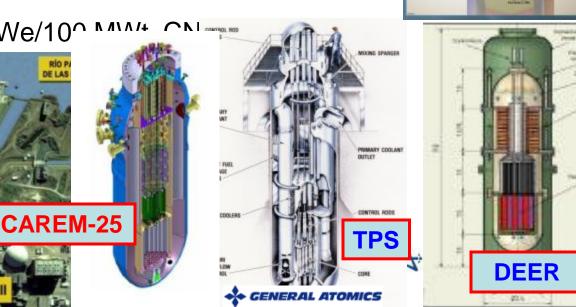
NHR-5: 5-MWt integral heating reactor at INET, Tsinghua U., Beijing, 1989 (with help from Germany)



- NBR (150 kWe) & SNB25: 2.5 MWe/25 MWt, RMC Kingston, unpressurized, 95° C outlet, refrigerant ORE
- **DEER:** 10 MWe/40 MWt Deployable Electric Energy Reactor, Radix Power & Energy
- TPS: 16.4 MWe/64 MWt TRIGA Power System **SNB-25 General Atomics**

DE LA

CAREM-25: 25 MWe/100 M/M/4 CN



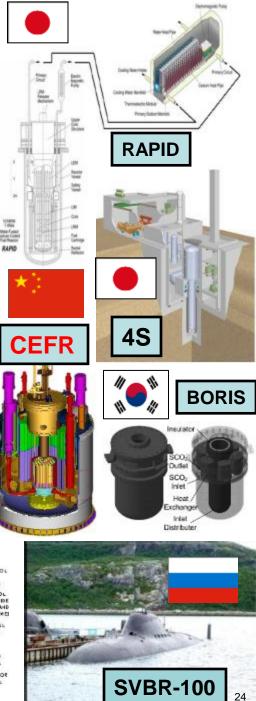
NHR-5

C Primary heat exchanger @ Containment SNB25 NUCLEAR REACTOR

Some Liquid Metal Cooled SMRs

Units				
Concept	Developer	Power	Coolant	Lifetime (years)
RAPID Refuelling by All Pins Integral Design	CRIEPI, Japan (1 MWe 10 MWt	Na	10
4S Super-Safe Small & Simple	Toshiba, Japan	10 MWe/30 MWt	Na	30
BORIS Battery Omnibus Reactor Integral System	SNU, Republic of Korea	10 MWe/22.2 MWt	Pb	20
CEFR Chinese Experimental Fast Reactor	CIAE, China/OKBM	20 MWe/65 MWt	Na	-
SVBR-10 Lead-Bismuth Fast Reactor	AKME-Engineering, Russia	12 MWe/43.3 MWt	PbBi	15-20
SSTAR Small Sealed Transportable Autonomous Reactor	ANL	20 MWe/45 MWt	PbBi	30
G4M Gen4 Module (HPM Hyperion Power Module)	Gen4 Energy Inc. (Hyperion Power Generation Inc.)	25 MWe/70 MWt	PbBi	10





He gas-cooled SMRs

- Pebble bed reactors
 - Germany AVR 15 MWe/46 MWt, 1967-88
 - China HTR-10 10-MWt, Tsinghua U., 2000
- Prismatic block reactors



- Japan HTTR 30 MWt test reactor, 1999 General Atomics **PS-MHP**: Remote Site
- General Atomics RS-MHR: Remote Site MHR 10-25 MWe
- Gas-cooled fast reactors
 - EM²: General Atomics 40 MWe/200 MWt
 <u>30 years</u> without refuelling

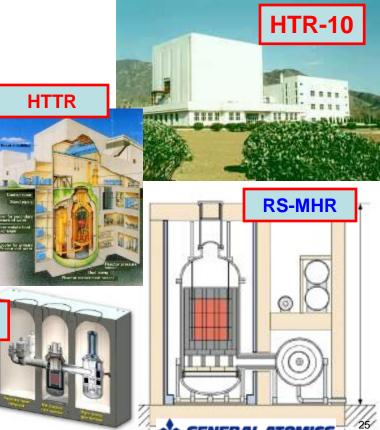


ALLEGRO

ALLEGRO: 50-80 MWt GFR prototype
 EURATOM/CEA, Czech Republic,
 Slovakia, Hungary

🚓 GENERAL ATOMICS



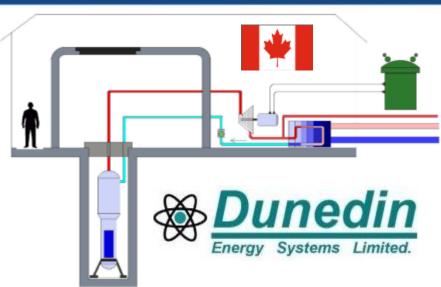


EM²

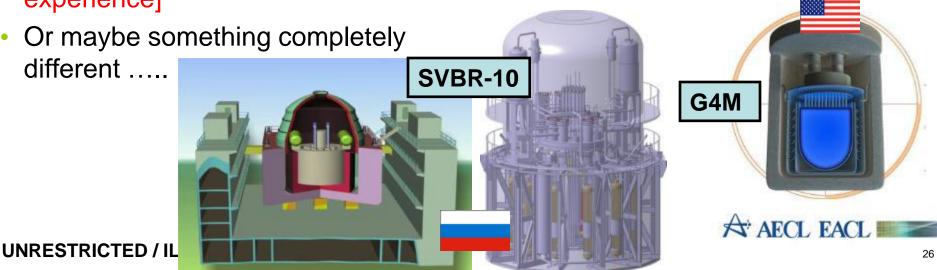


The future: What the Canadian very small reactor market might involve

- Compact <u>IPWRs</u> derived from Marine/Naval reactor technology
- Scaled down integral Pb-Bi <u>LMRs</u> also linked to Marine/Naval reactor technology [80 reactor-years of experience]



5-MWe SMART (Small, Modular, Adaptable, Reactor Technology)



What we really need: The 'i-Reactor' concept

(How do you like them Apples?)

- Must be compact, light & licensed to be <u>transportable</u> with containment & shielding
 - Boeing Chinook 234: Highest rated commercially certified lift capacity at 27,000 lbs/12.2 Mg
- Must dissipate decay heat passively to ambient air at all times
- Must be licensable for <u>autonomous reactor operation</u>
- This suggests a very low-power (<1 MWe), compact gas-cooled fast reactor
 - Pressurized He primary coolant: inert, light, ~550° C
 - ->50-year core life without refuelling
 - -<20 wt% ²³⁵U LEU fuel; probably Be, BeO neutron reflector
 - Desirable to have essential loads powered by TEs
- <u>Uncertain technical/economical feasibility</u>
- Requirements similar for a Lunar Base reactor
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Mobile Nuclear Power for **Future Land Combat**

BY MARVIN DAKER SCHAPPER and INE CHANG

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Mobile Nuclear Reactors Could Provide Power and Jet Fuel for Military, DARPA Says

By Jeremy Hau Posted 03.51 2010 at 12:37 pm III 14 Comments



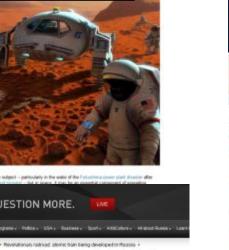
Navy Aircraft Cauld nuclear-powered carriers use their reactors and seawater to provide jet fuel for their aircraft? U.S. NevyStephen Rose

Making U.S. Navy carrier groups and Army bases more self-sufficient and energy-efficient could mean turning to mobile nuclear reactors. The Pentagon's DARPA scientists have put forth the modest proposal of deploying miniature reactors to convert hydrogen and carbon into military jet fuel, as well as providing power, The Register reports.

That plan could fit well with the U.S. Navy's "Green Strike Group" concept for biofuel and nuclearwarad secole. The Renieter nainte with that nuclear, nowated simret carrier music moles use

Recent interest in mobile reactors





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TAGS: Nation. Scillett. Gamus. Yenkles



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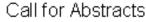
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Wrap-up

- A potential niche market exists for very small reactor units in remote Canadian locations, especially to support <u>mine development</u>
- Very small reactors are technically doable, dozens of SMR concepts exist (especially from Russia), many are of Marine/Naval origin
 - Canada once was a leader in this field with AECL's SES-10 & Nuclear Battery programs
- The very small reactor market might be addressed using existing technologies, but economic viability would likely require
 - –<u>Post Fukushima</u>: guaranteed passive decay heat removal to ambient air & no hydrogen (or steam) explosion risk
 - -Licensed for *autonomous/unattended* operation of the reactor core
 - -<u>**Transportability</u>** of the nuclear portion as a sealed unit (new & used)</u>
 - –A large # of application units (>100) to be worthwhile; need a good technical fit with the Alberta Oil Sands Western Focus

-<u>Social acceptance & a compelling business case/value proposition</u>



2nd International Technical Meeting on Small Reactors

2012 November 7-9 The Albert at Bay Hotel , Ottawa, Ontario CANADA

"Celebrating NPD's 50th Anniversary"

*O bjectiv*e

952 - 201

Atom b Evergy of Canada Limited (A,ECL) is losting the 2^m international Technical Meeting on Small Reactors. There is growing international interestand actually in the development of small inclear reactor technology. This meeting will provide participants with an opportunity to share ideas and exchange information on New C

This Technical Meeting will couer topics of Interest to designers, operators, researchers and analysis involved in the design, development and depolyment of small reactors for power generation and research. A special session is planned to focus on small modular reactors (SMR) for generating electricity and process heat, particibility in small grids and remote locations. On the last day of the Technical Meeting (November S¹⁶), AECL will lost a for root the Chaik River Laboratories for all interested attendees. The for rwill include the ZED-2 and NRU reactors.

Following the success of the first Technical Meeting in Noumber 2010, which captured numerous accomptishments of low-power official facilities and small reactors the second Technical Meeting 6 dedicated to the ack Evenents, capabilities, and future prospects of small reactors. This meeting also celebrates the SU⁶ An whersary of the Nickear Power Demonstration (NPD) reactor which was the first small reactor (20 MW/e) to generate electricity in Canada.

Topics of Interest

Presentations related to the following topics are of interest. To this Technical Neeting:

- Safety and Libensing
- Reactor P lysts (pliysts code us idation, bias and uncertainty, benchmarking, etc.)
- Aduanced Fuels (new compositions, interently sate fuels, etc.)
- Instrumentation and Control
- Thermalitydravlics (passile safety, heat pipes, etc.).
- Research and Test Reactors
- Commercial SMRs to rejectricity generation
- Small reactors for remote locations
- NouelConcepts

U

Autonomous Control and Operation

Abstract Submission

Actions should submittae extended abstract (world three pages) with contact holm atton, us electron binall, to the Teichical Program Chair, Showel Yoe, (<u>res@acolca</u>). Extended abstracts will be published on CD in the Conference Proceedings.

Technical Meeting Organizers

Steering Committee	talı Harry, CIC
	John Root, CCNI
I I I I I I I I I I I I I I I I I I I	Rom key Duffey, DSM Associates
	Marcel de Vos, CNSC
	PaulLabbe, DRDC
	Dan Meneley, UOIT
	John Mickenzle, Sask Power
Gene BIChat	Dauld Sears
Te chilical Program Chair	ShuwelYue

Key Dates

Ertended abstracts deadline	
Early-Blid registration deadline	September 15, 2012

Further Information

Additional Information may be obtained by ulsiting www.aeci.ca/SUR or by contacting Technical Program Chair, Shuwel Yite, ABCL, Chaik River, Labotatories, Chaik River, Ontario KOJ 1JD CANADA, Tet (513) 584-3311 ext. 44635; Email: <u>vies@aeci.ca</u>.





- 2012 Nov. 7-9, Ottawa
- contact yues@aecl.ca



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